

Radiation Damage Calculations for the Rare Isotope Accelerator

Susana Reyes, Brian D. Wirth,¹ L. E. Ahle, W. Stein
and our RIA target area R&D collaborators

Lawrence Livermore National Laboratory

¹ University of California, Berkeley

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Presentation overview

- **Motivation: need to address radiation damage of components in high power accelerator facilities such as RIA**
- **Overview: ion interactions with materials, radiation damage and a review of irradiation effects in Cu and Al**
- **Radiation damage simulations for RIA beam dump concepts**
- **ATLAS simulations and experiments**
- **Summary & suggested future directions**

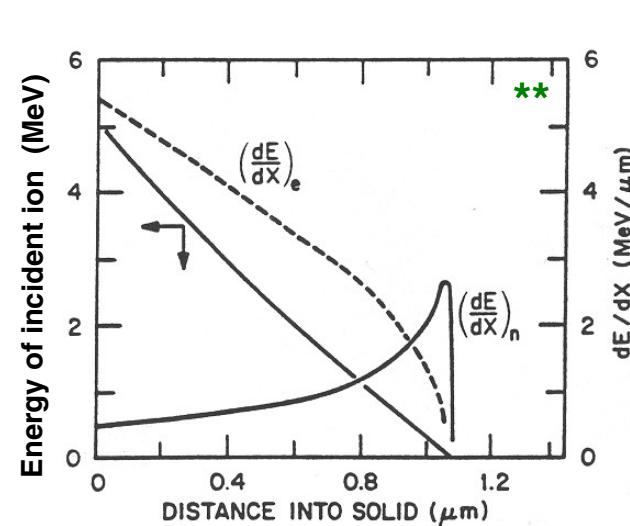
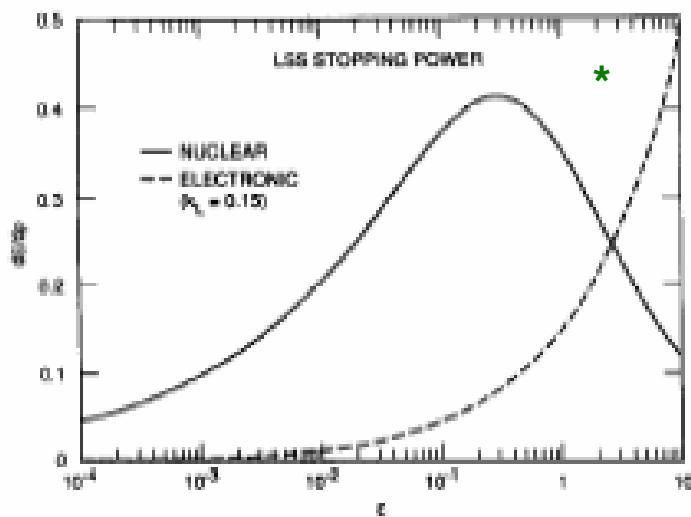
Introduction

- In the context of high power accelerator facilities such as RIA, high priority is given to optimization of component lifetime, so as development of remote handling/maintenance systems required for radiation damaged and activated components
- Radiation damage is a critical issue for those components with structural, electrical, and magnetic functions (i.e. target structures, beam dumps, beam windows, catchers, magnets, insulators and seals)
- Experimental data and previous operating experience are extremely useful when available, however for next-generation facilities, simulations become essential to predict material behavior under irradiation

Overview of ion interaction with materials

- Energy loss (slowing down) of energetic ions in a solid is characterized by the linear energy transfer rate or stopping power, $S(E) = dE/dx$; commonly divided into energy lost in electronic excitation and through atomic collisions (nuclear interactions)

$$S(E) = \frac{dE}{dx} = \left. \frac{dE}{dx} \right|_{\text{elec}} + \left. \frac{dE}{dx} \right|_{\text{nuclear}}$$



- Except for large electronic energy loss rates ($dE/dx|_{\text{elec}} > \sim 10 \text{ keV/nm}$), this energy dissipates as heat with no lasting damage production

* M.T. Robinson, *JNucMat* **216** (1994) 1.

** D.R. Olander, Fundamental Aspects of Nuclear Reactor Fuel Elements.

Overview of ion interaction with materials (cont)

- Nuclear interactions, predominately as binary elastic collisions, produce permanent atomic displacements - in the form of vacancy + self-interstitial atom = Frenkel pair. The number of displacements depends on the kinetic energy transfer in the collision, T, the displacement energy, E_d , of the solid, and the particle flux, Φ

$$\frac{dpa}{s} = \int_{E_d/\Lambda}^{\infty} \sigma_d(E) \phi(E) dE \quad \Lambda = \frac{4m_1 m_2}{(m_1 + m_2)^2}$$

$$\sigma_d(E) = \int_{E_d}^{\Lambda E} \sigma(E, T) v(T) dT \quad v(T) \equiv \# \text{ of displacements produced by recoil of energy } T \\ = \xi(T, Z) \frac{T}{2E_d}$$

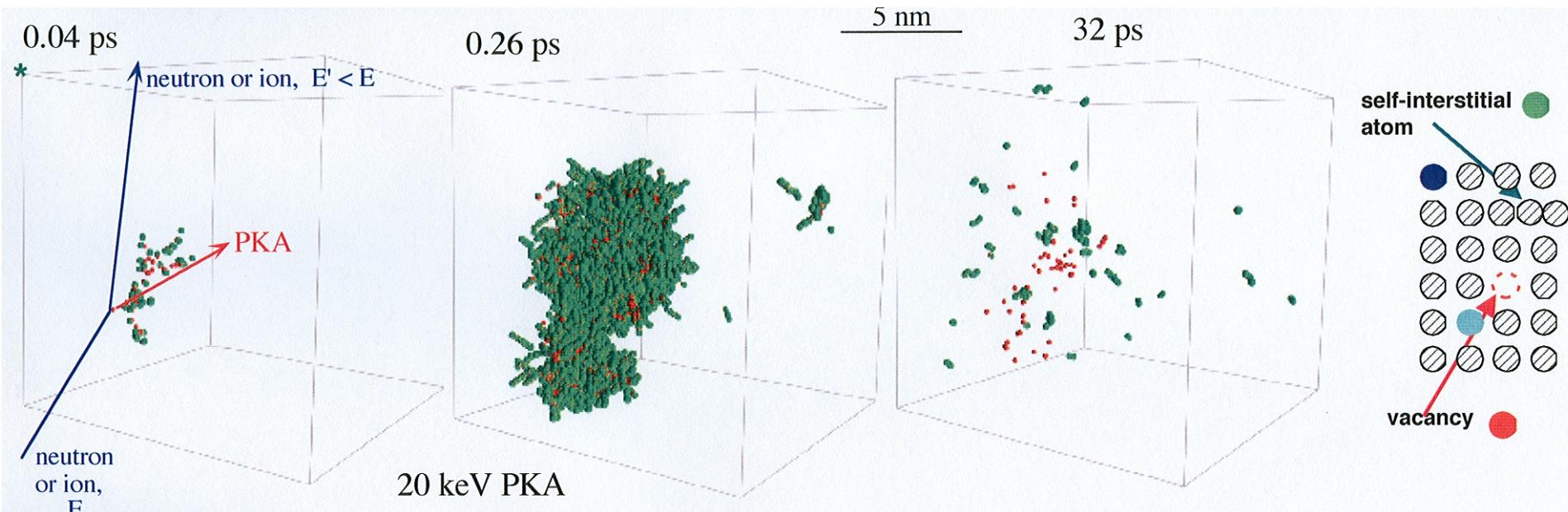
- The high-energy particle transport codes, including SPECTER, PHITS & MARS, use these expressions to calculate displacements - accuracy depends on the differential scattering cross-sections and treatment of energy partitioning (ξ) between electronic & nuclear stopping

* M.T. Robinson, *JNucMat* **216** (1994) 1.

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Radiation damage of materials

- Radiation damage is initiated by Frenkel pair production in collisional cascades

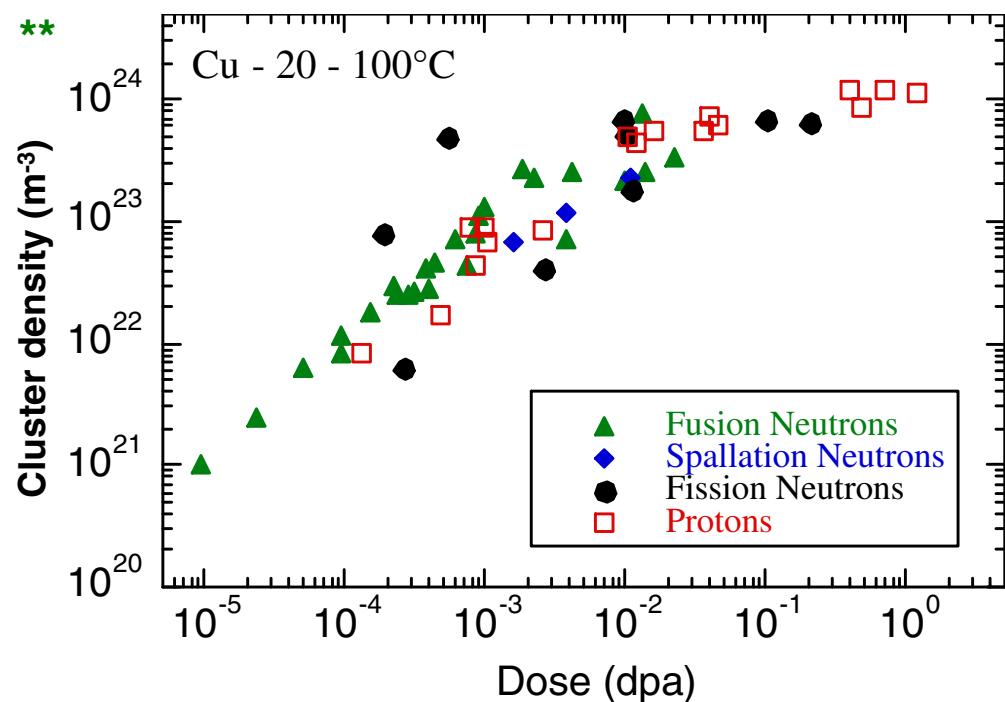
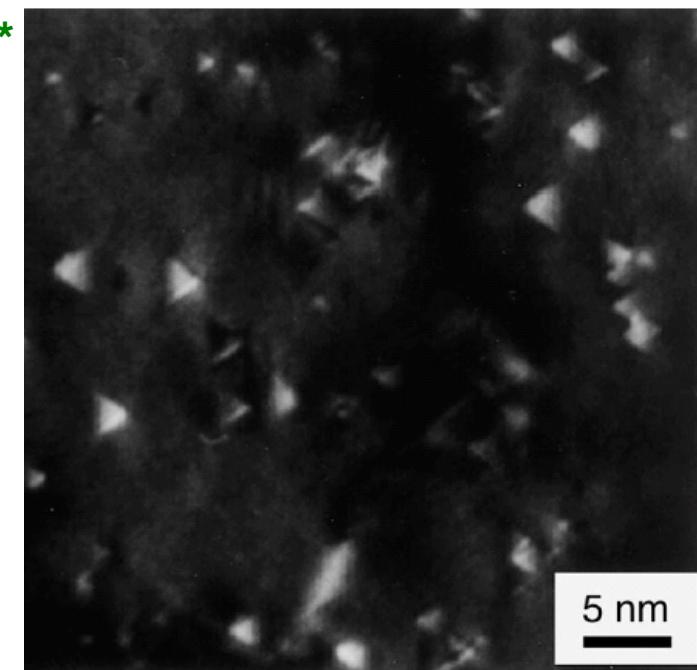


- Defect diffusion, clustering and solute enhanced diffusion, precipitation and segregation (in alloys) responsible for microstructural evolution
- Defect clusters and microstructural defects impede dislocation motion (increased strength and decreased ductility) and scatter phonons (increased electrical resistivity, decreased thermal conductivity)

* R.E. Stoller, personal communication.

Microstructure of irradiated Cu

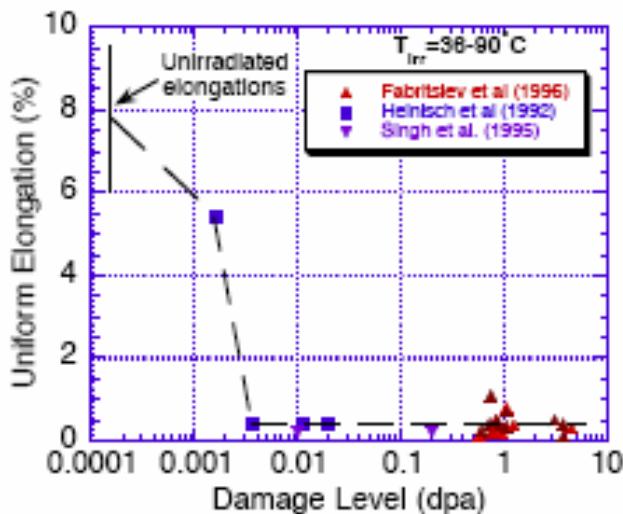
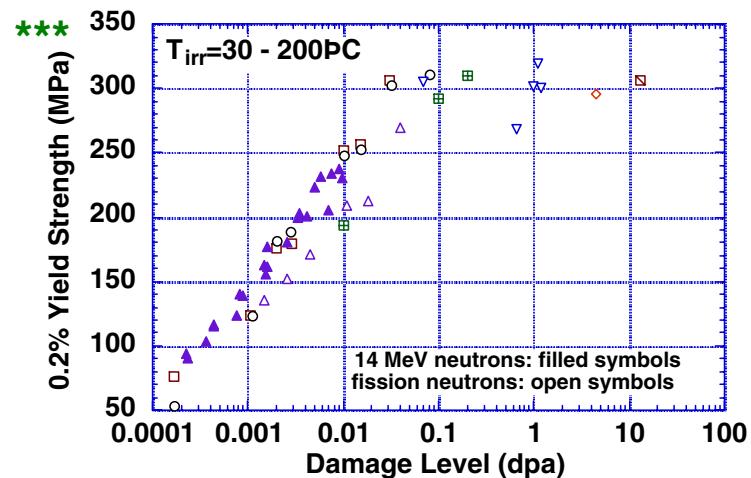
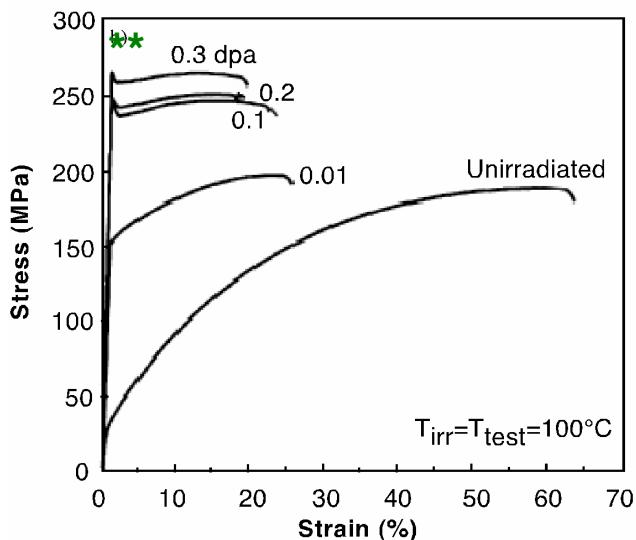
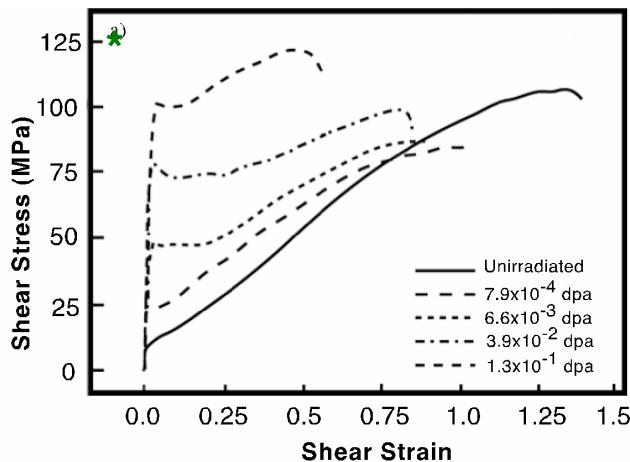
- Large number density of nanometer-sized defect clusters commonly observed in irradiated metals. In Cu, these defect clusters are predominately small (≈ 2.5 nm) stacking fault tetrahedra (SFT) and dislocation loops. Measurements of Cu irradiated at $T \approx 100^\circ\text{C}$ indicate saturation of defect cluster density with increasing fluence



TEM observations of SFT cluster density indicate saturation of defect density by about 1 dpa

Mechanical behavior of irradiated Cu ($T_{irr} \approx 100^\circ C$)

- Radiation-induced defect clusters serve as obstacles to dislocation motion, thereby strengthening the material, but also decrease strain to failure in tensile test

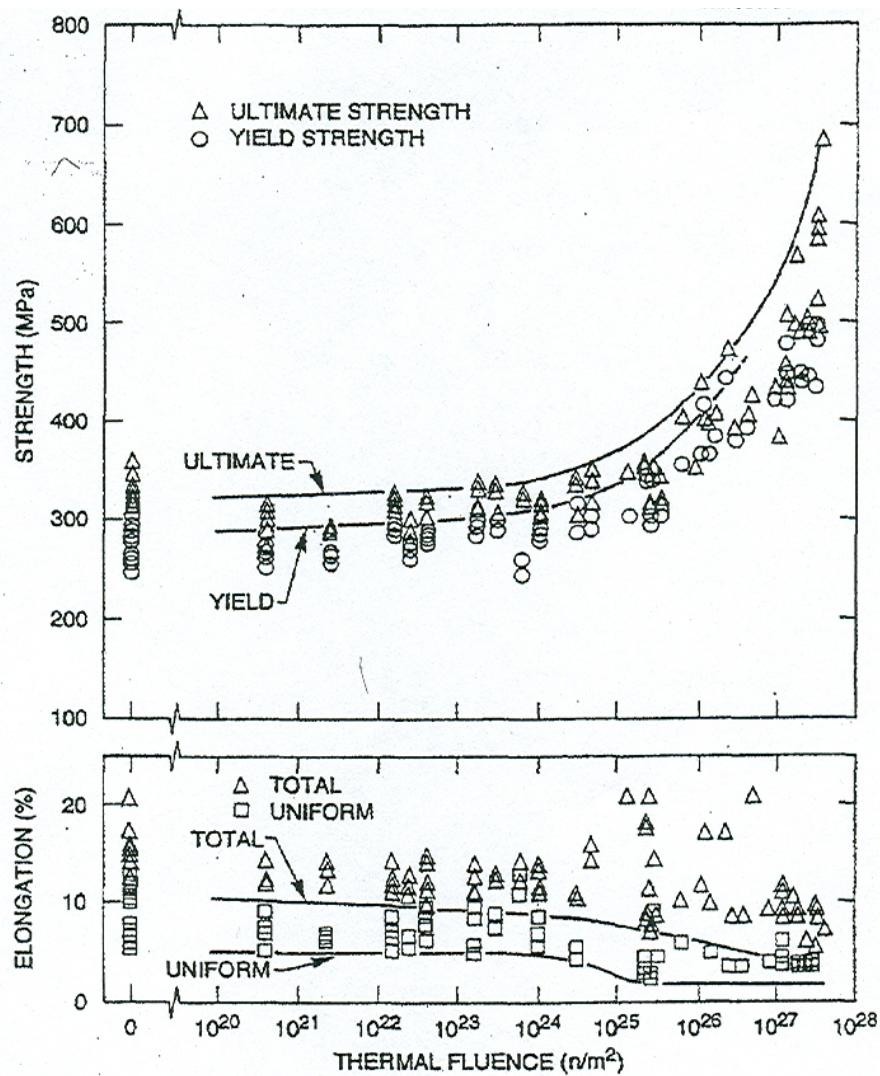


* Y. Dai, et al., *J. Nucl. Mat.*, **212-215** (1994) 393..

** B.N. Singh, D.J. Edwards, and P. Toft, *J Nucl Mat*, **299** (2001) 205, *** S.J. Zinkle, personal communication.

Mechanical behavior of irradiated Al ($T_{irr} \approx 100^\circ C$)*

- Similar effects observed in pure Al and aluminum alloys. While the total elongation (ductility) approaches saturation, the strength does not.



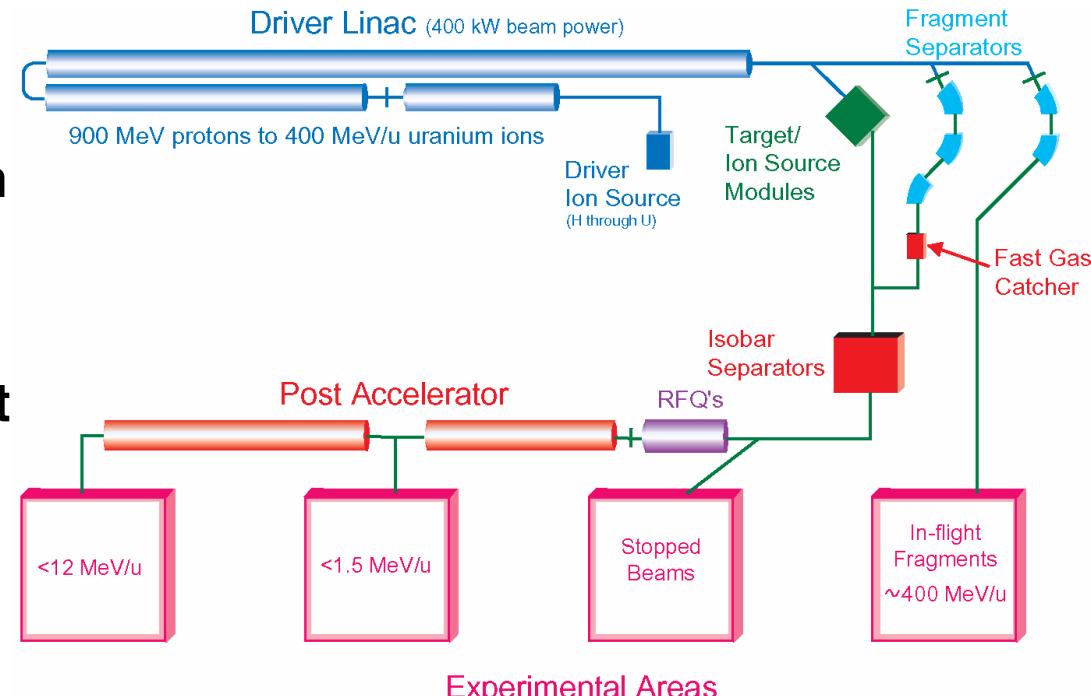
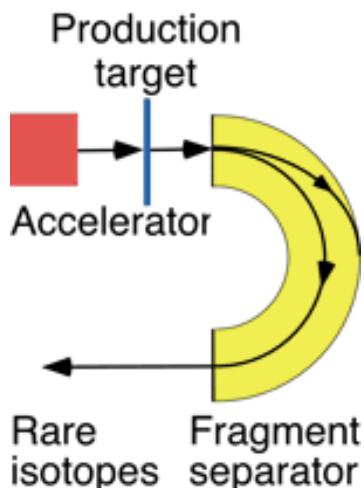
Swift Heavy Ion effects

- Numerous studies* have shown strong effects on damage production (# defects produced and defected microstructure) by ***Swift Heavy Ion (SHI)*** irradiation when $(dE/dx)_{elec} > \approx 10 \text{ keV/nm}$.
 - Effects vary from damage annealing (reduced defect densities) at lower $(dE/dx)_{elec}$ to additional damage creation and more rapid radiation enhanced precipitation kinetics at higher $(dE/dx)_{elec}$.
- Detailed SHI mechanisms are not well established. Two main models:
 - ***Thermal spike model:*** excited electrons rapidly transfer energy to phonons. Very large energy (heat) deposition leading to localized melting & rapid cooling → high defect densities or amorphization.
 - ***Coulomb explosion model:*** large positive space charge resulting from electronic excitation leads to strong atomic repulsion, atomic displacements and a cylindrical shock wave.
- Experimental irradiations with high-energy U ions needed to resolve SHI concerns.

* e.g., A. Dunlop, D. Lesueur, and A. Barbu, *J. Nucl. Mat.* **205** (1993) 426., A. Dunlop et al., *Nuc. Inst. Meth. B.* **90** (1994) 330., H. Dammak et al., *Phys Rev Lett* **74** (1995) 1135., Z.G. Wang et al., *Nuc. Inst. Meth. B.* **115** (1996) 577.

RIA overview

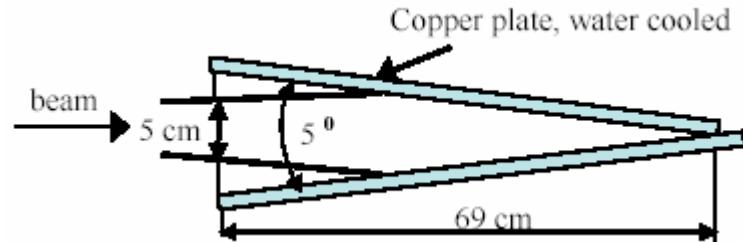
- RIA is the next-generation national user facility for basic and applied research with radioactive beams
- RIA will have one or more fragmentation stations that will accept up to 400 kW beams of heavy ions



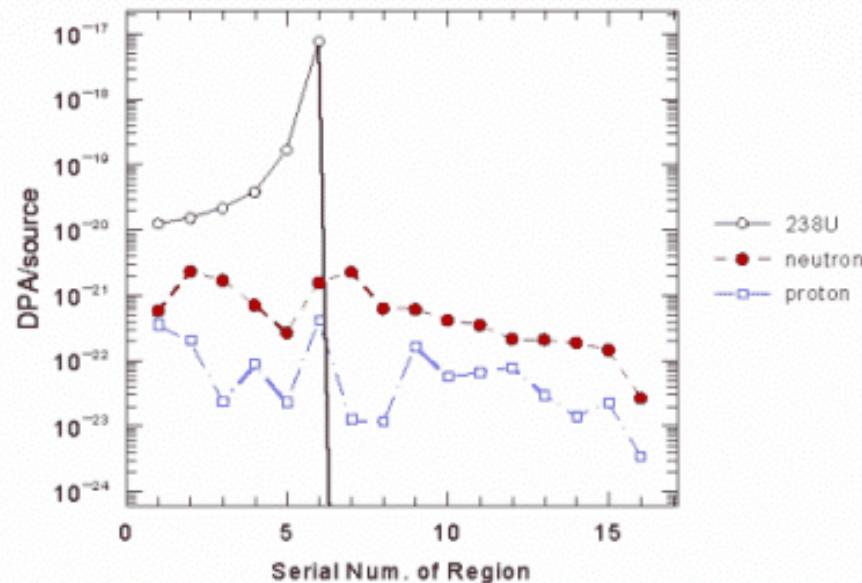
- Designing a fragmentation beam dump for RIA is a critical challenge:
 - must be compact to reduce optical aberrations
 - must survive deposition of hundreds kWs of power
 - mechanically simple to facilitate remote handling
 - must survive radiation damage by primary beam

Simulations for RIA beam dump

- Developed static water-cooled wedge-shaped beam dump for a 320 MeV/A U beam at 400 pnA/cm², 5 cm² spot

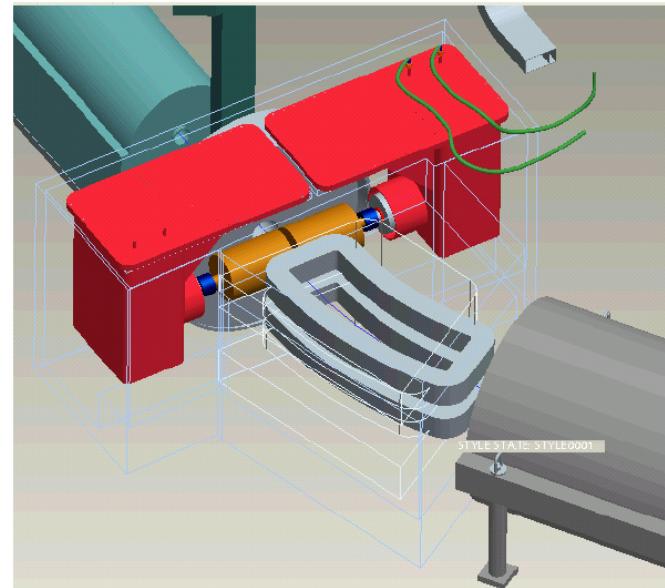
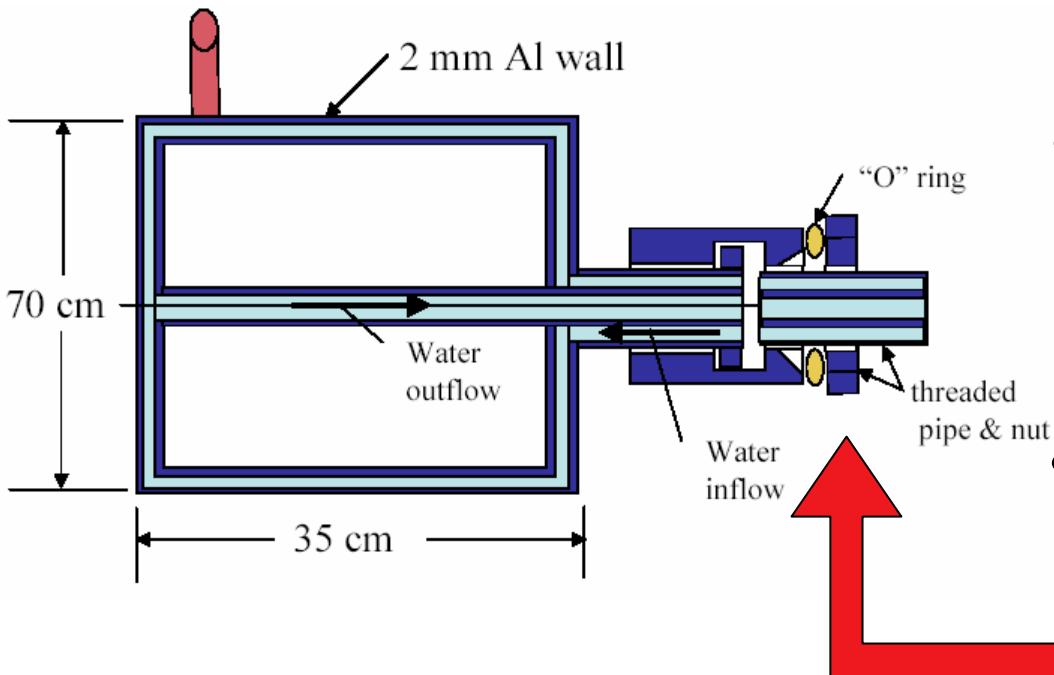


- Beam sputtering was shown not to be significant
- However, radiation damage predictions using PHITS transport code indicated very high levels of DPA and gas transmutants in copper:
 - maximum DPA ~25 DPA per full power day
 - several hundreds appm H and He gas transmutants
 - effects of SHI irradiation and relationship between DPA and properties degradation needs to be addressed



Advanced beam dump design

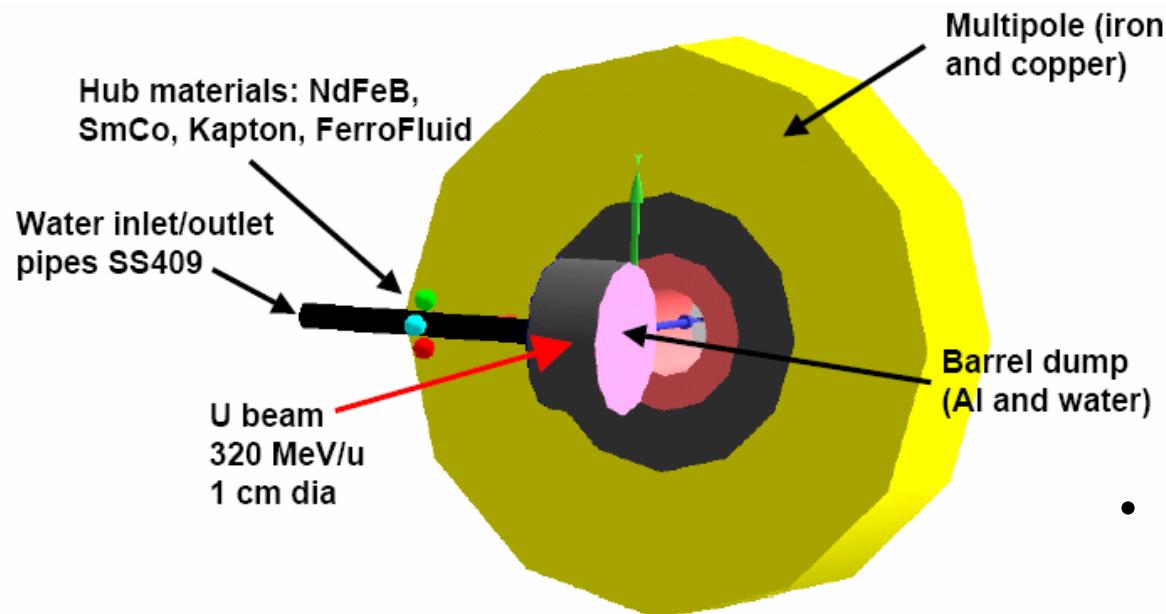
- To mitigate radiation damage, rotating beam dump concepts are being considered
- In particular, a rotating barrel-shaped dump has been designed capable of withstanding a 1cm-diameter beam spot



- U beam slows down in cooling water, avoiding high DPA values in structural material
- Lifetime increases significantly, however more complex design (i.e. rotating vacuum seal)

PHITS simulations for rotating beam dump

- The heavy ion transport code PHITS to simulate particle transport in pre-separator area
- Model includes barrel beam dump, steel water inlet/outlet pipes, rotating vacuum seal with representative materials, and downstream multipole magnet



- Assumed operation with a 320 MeV U beam with 1 cm-diameter spot size at a current of 3×10^{13} pps

PHITS results

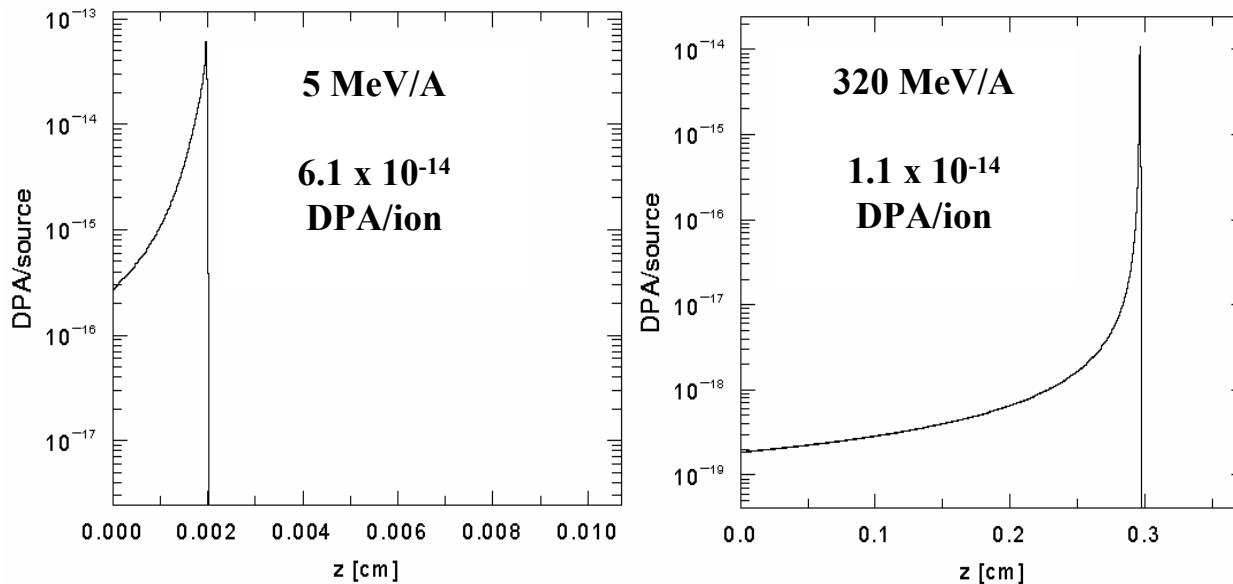
Material	Density (g/cc)	Effective dose (MGy/yr*)	Dose limit (MGy)	DPA/yr*
NdFeB	6	0.29	0.1	4.5E-06
SmCo	8.82	0.15	100	5.9E-06
Kapton	1.42	0.74	10	7.6E-07
FerroFluid	1.42	1.08	>1?	7.1E-07

*Assumed that fragmentation line is operating at full power for one-third of the calendar year

- DPA in the rotating seal materials found to be negligible
- Maximum DPA rate in the Al barrel ~0.03 DPA/yr (most of the primary beam stops in water); maximum DPA in multipole ~ 5×10^{-4} DPA/yr
- Gas production in Al barrel is not an issue
- However, with expected large electronic energy loss rates ($dE/dx|_{\text{elec}} > \sim 10 \text{ keV/nm}$), SHI effects still need to be addressed

ATLAS Experiments

- Results from PHITS show DPA unacceptable for static beam dump, but allowable for the rotating beam dump case
- Some issues still not addressed by PHITS simulations:
 - What is the impact of SHI effects?
 - How does DPA value translate into effects on material properties?
- Recent experimental campaign at ATLAS (6 MeV/A) to help find these answers (see J. Nolen's presentation)



Summary

- High-energy particle irradiation of Cu at low ($\approx 100^{\circ}\text{C}$) temperatures, produces significant radiation damage, with yield stress increases of ≈ 300 MPa, uniform elongations $< 1\%$ and thermal k decreases of 5 - 10%
- Radiation effects in Cu saturate at doses of ≈ 1 dpa (*limited high-dose (>10 dpa) data, behavior may change with high gas levels*)
- Preliminary analyses show high DPA levels in RIA Cu beam dump: experimental data under RIA conditions is needed
- A rotating beam dump design has been proposed that mitigates the problem of radiation damage from static design
- However, concerns specifically related to the RIA irradiation environment require additional research:
 - benchmarking of PHITS simulations with experiments/models
 - swift heavy ion (SHI) irradiation effects
 - underestimates of radiation doses due to limited cross-section data for high-energy electronic losses, nuclear scattering and inelastic collision processes