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Simulations in support of RIA Target Area R&D

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2nd High-Power Targetry Workshop
October 10–14, 2005 Oak Ridge, TN



RIA R&D Participants (now starting 2nd year of funded effort)

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U.S. Department of Energy



Office of Science
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Outline

- Ronningen
 - Examples of Simulations for Fragment Pre-Separator Area Pre-Conceptual Design
 - Quadrupole radiation damage simulations
 - Beam Dump simulations
 - Bulk Shielding
- Remec
 - Examples of Simulations for ISOL Target Area Pre-Conceptual Design
 - Two-step target simulations
 - Large-scale simulations examples



A Sampling of RIA Primary Beams

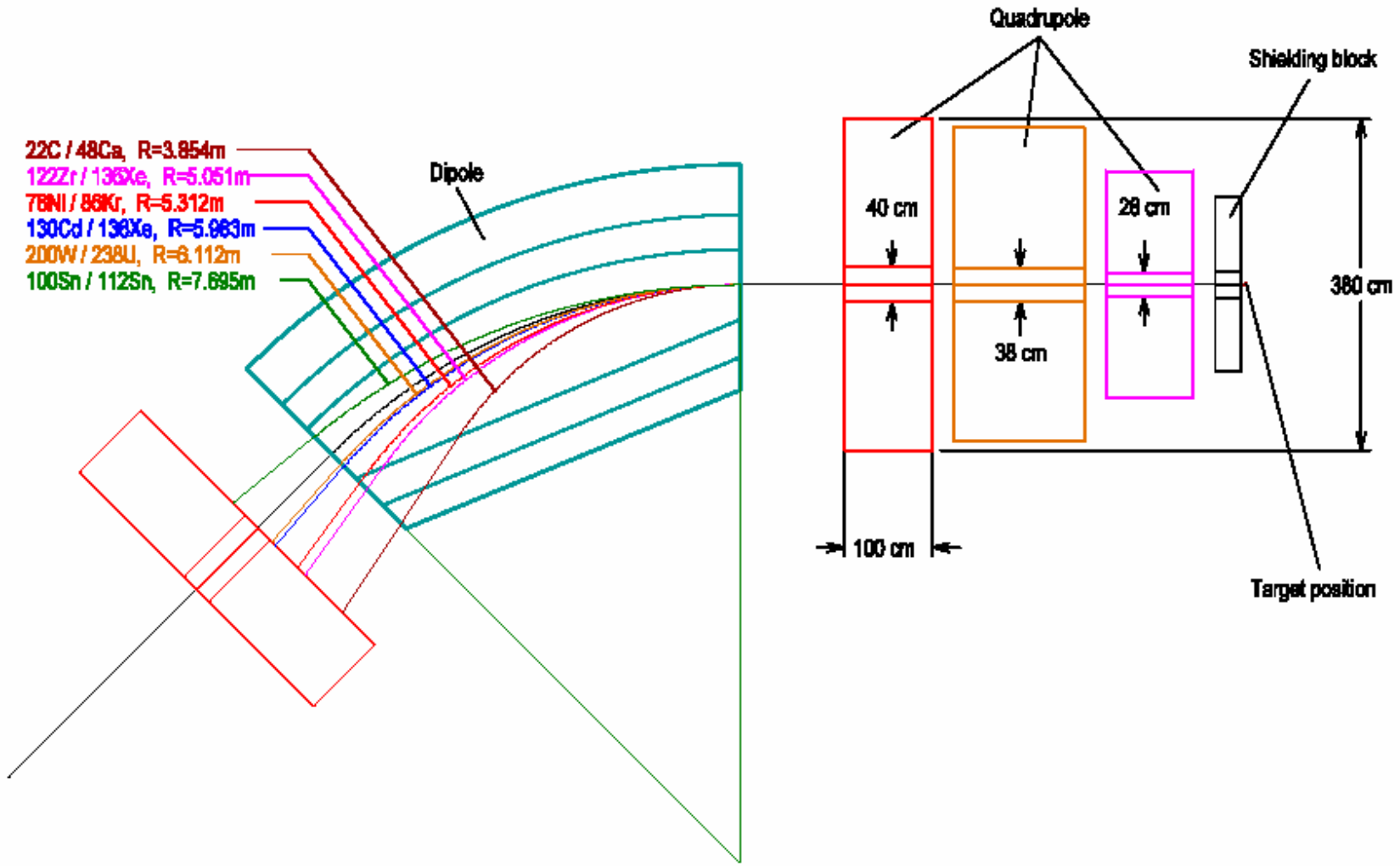
Current technology limits U to about 130 kW.

All the rest are 400 kW.

ION	A	Z	ENERGY (MeV/nucleon)
H	1	1	1019
³ He	3	2	777
D	2	1	622
O	18	8	560
Ar	40	18	566
Kr	86	36	510
Xe	136	54	470
U	238	92	400

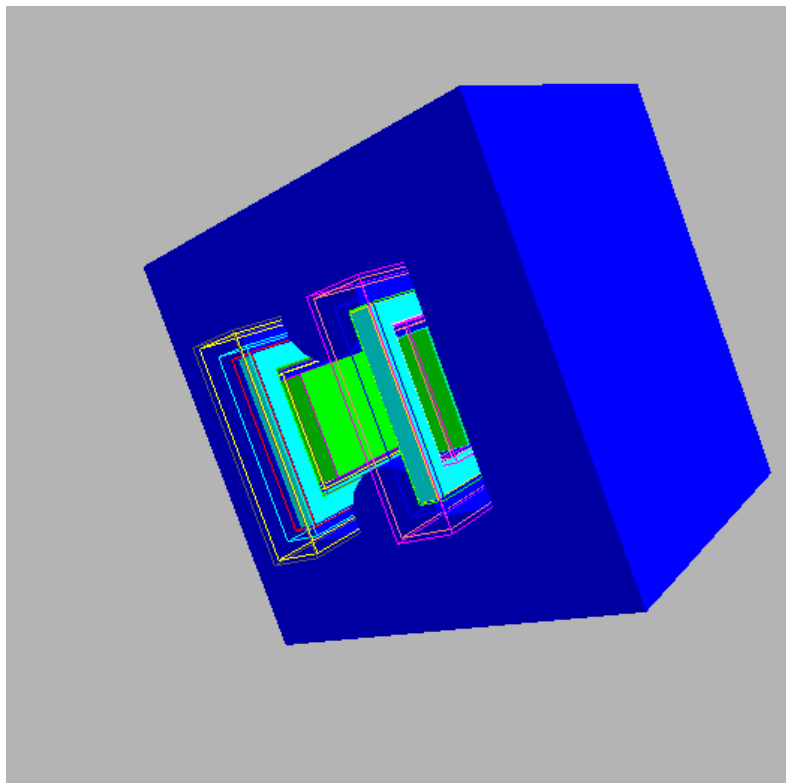
Where Do Primary Beam and Fragments Go?

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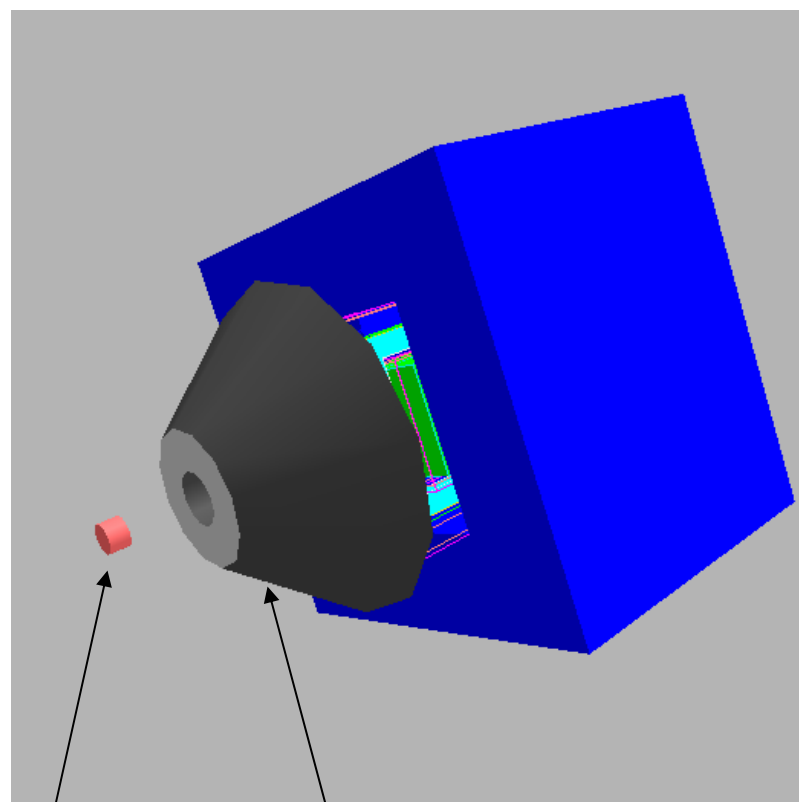


“Realistic” Quadrupole Geometry using BNL Design with Realistic Material Compositions

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Frames are cryostat walls
HTS Coil: Ag+BSCCO
Insulator: AlO+He

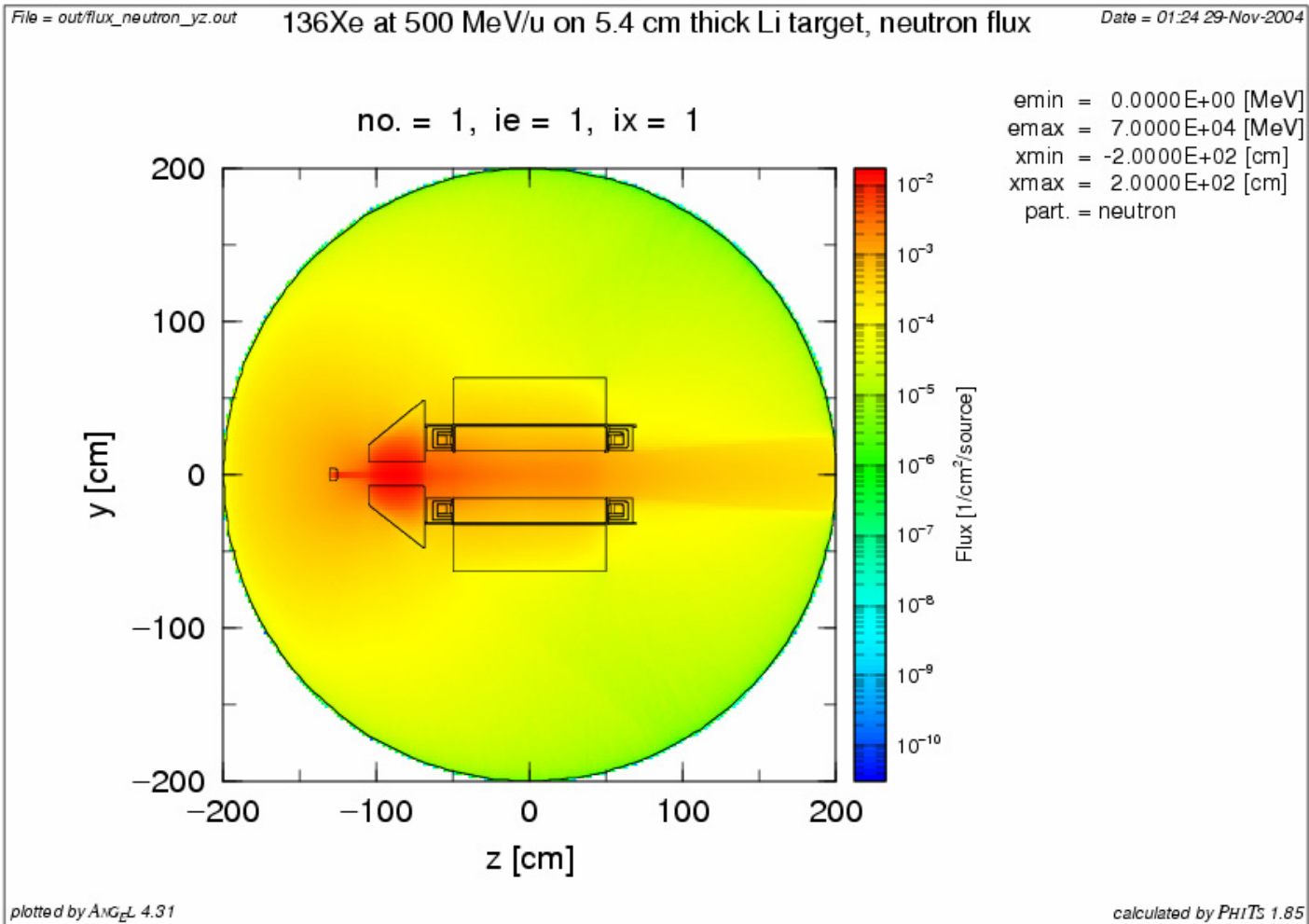


Target

Hevimet shield

Neutron Flux in Target, 1st Quad Area - No Magnetic Field

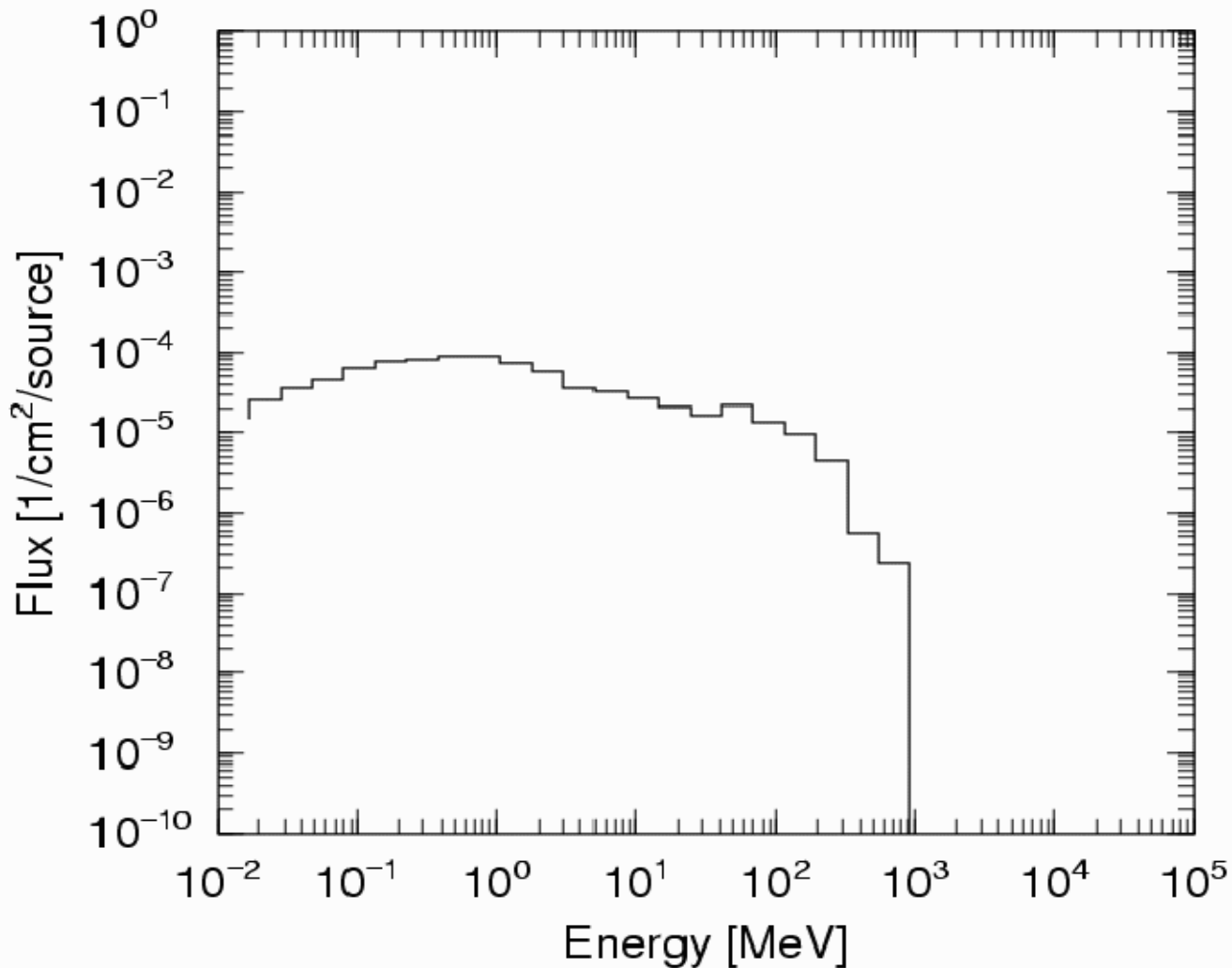
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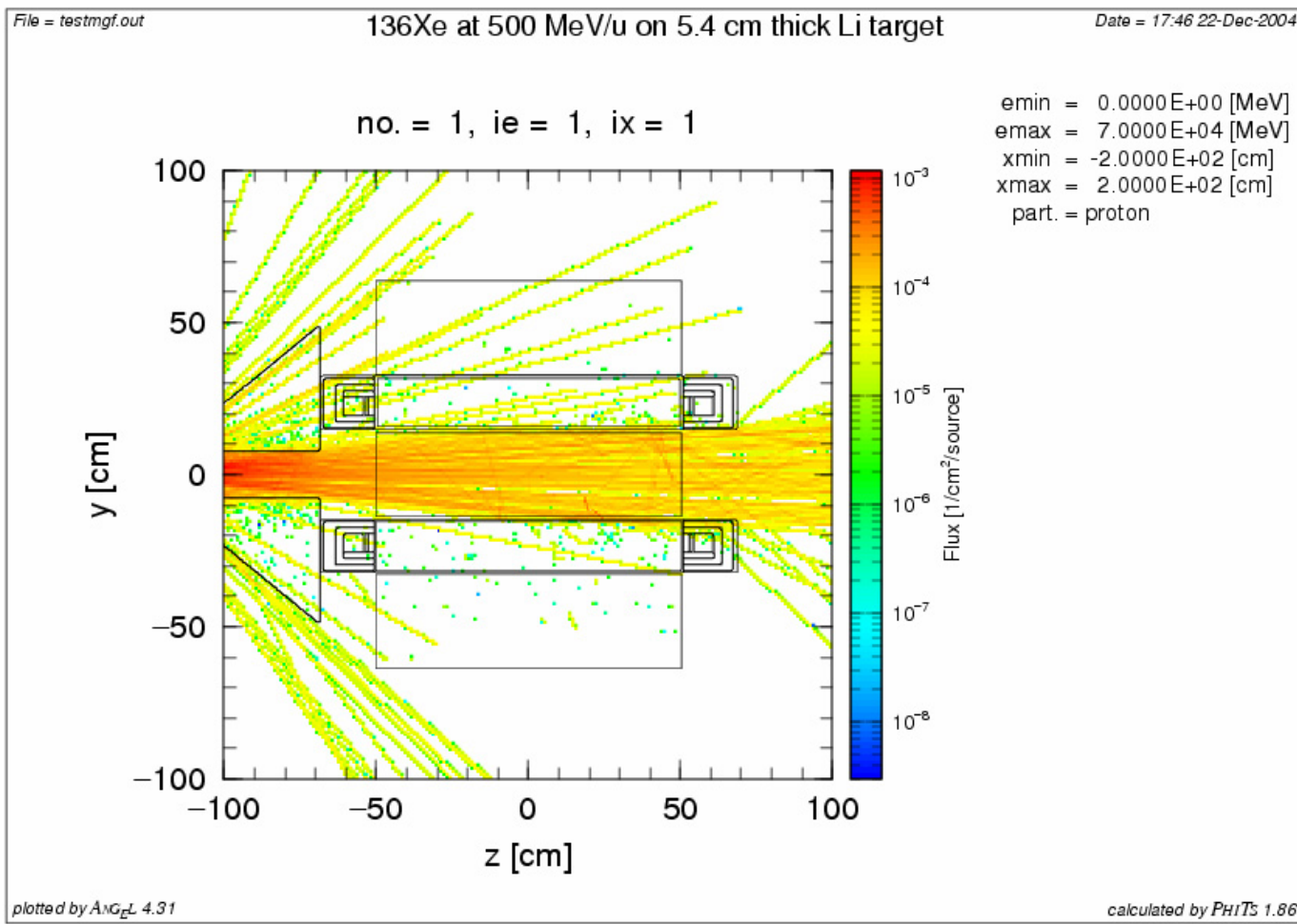
Neutron spectrum at the coil for ^{136}Xe at 500 MeV/u (PHITS simulation)

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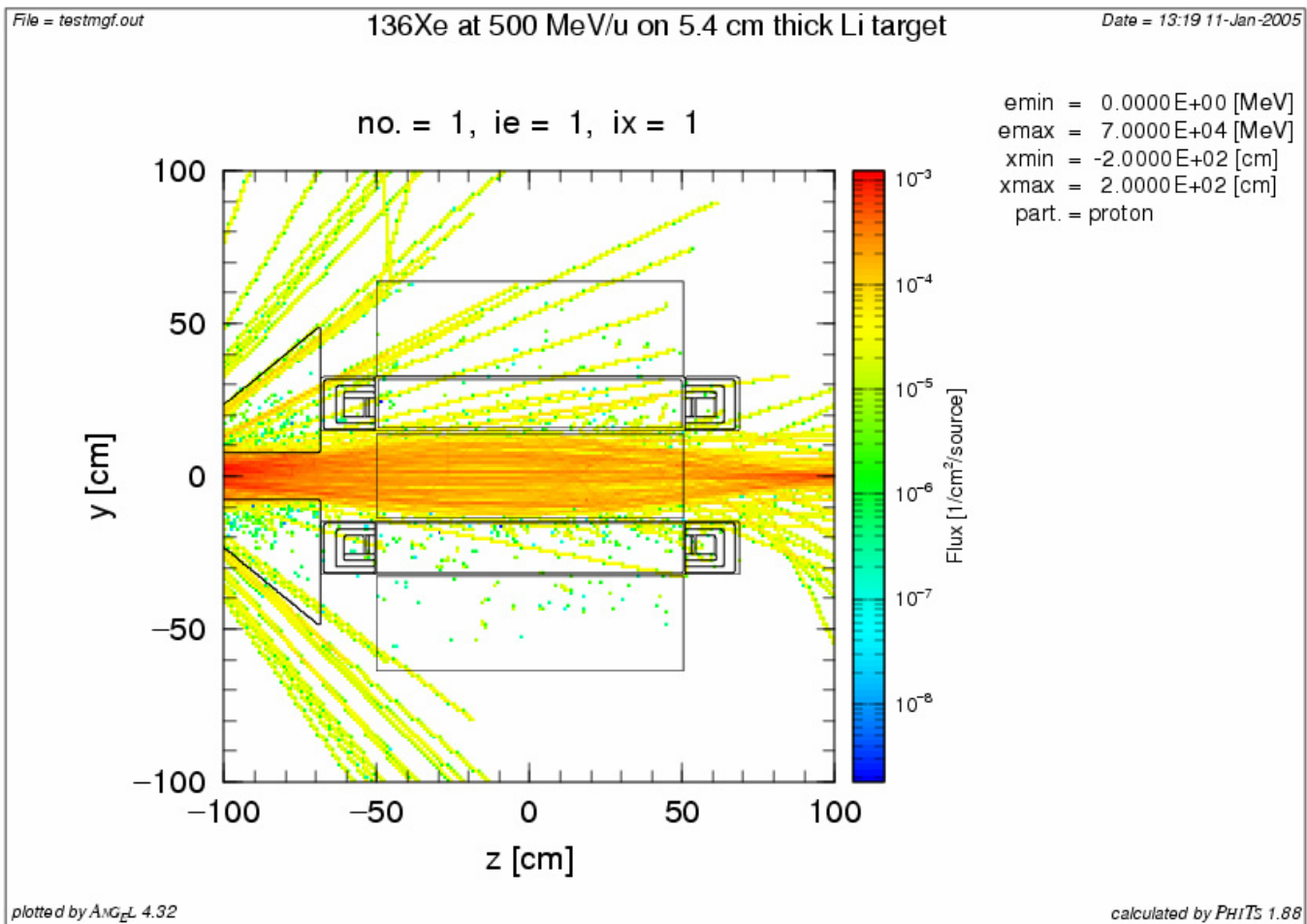
no. = 6, reg = 6



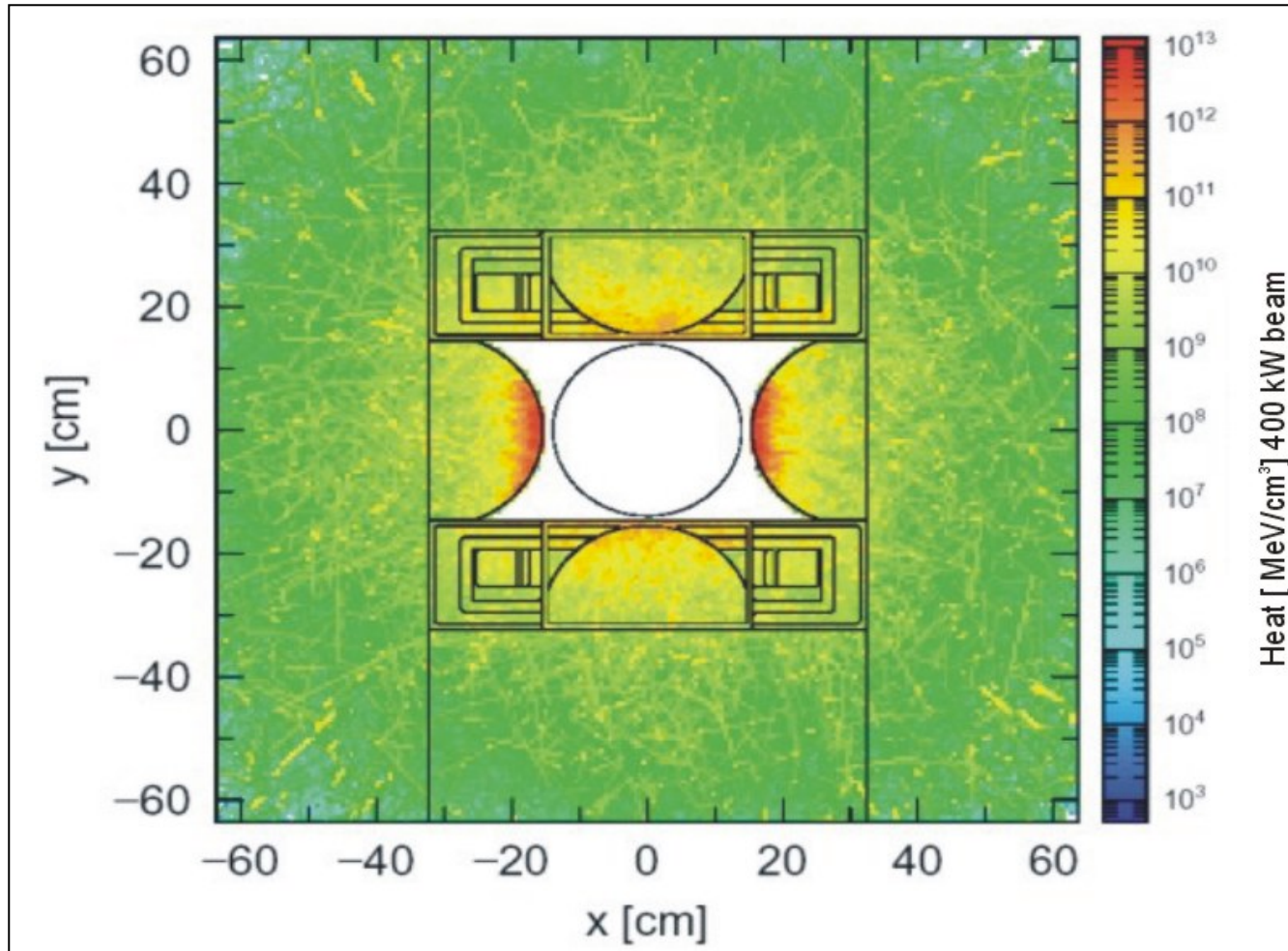
Protons – no magnetic field



Protons – Quadrupole magnetic field ON



Total Heat Tally



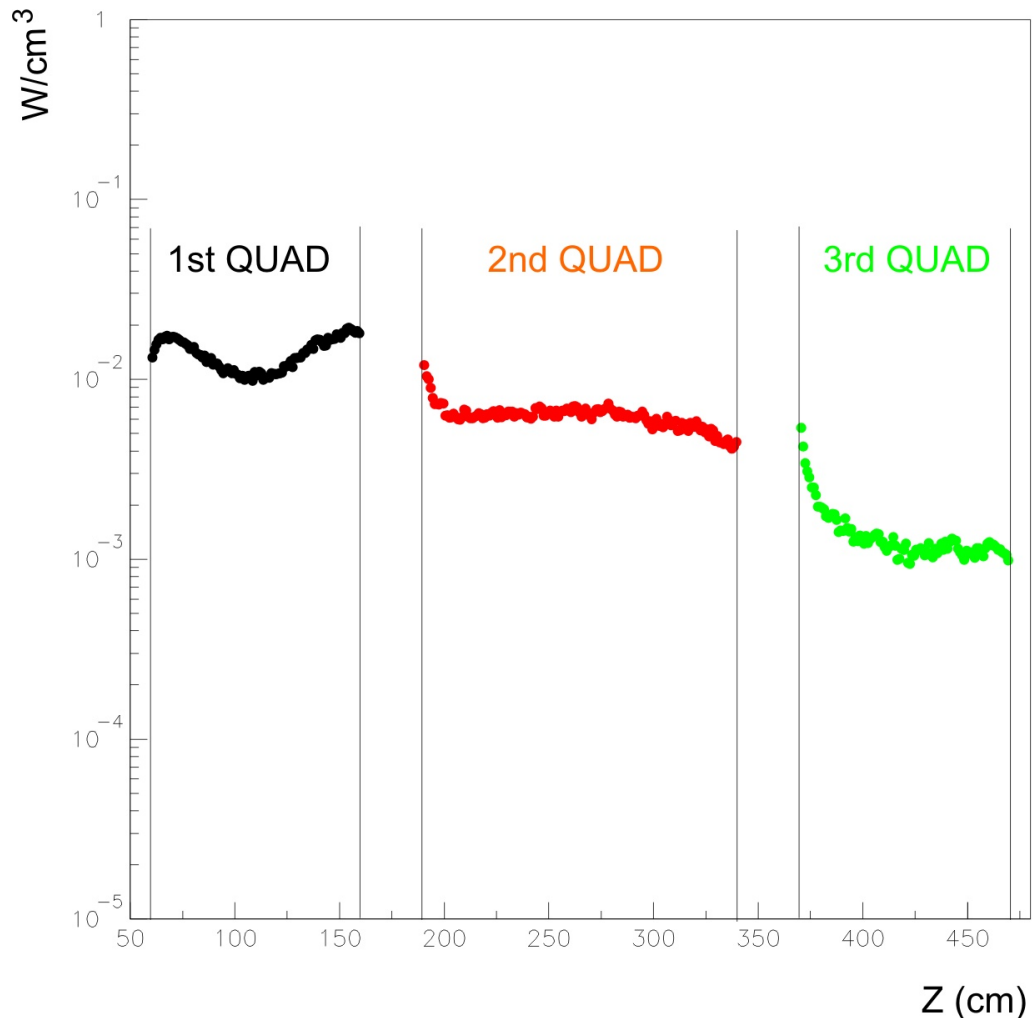
Triplet heat – no magnetic fields

- Like the peak magnetic field determining the conductor requirements, the **maximum dose in a single area determines the coil life time.**

- Note the dose is a **factor of two higher** than calculations done for a single quad. This is because of the enclosure reflecting **low energy neutron back into the coils.**

- The **peak doses** on each subsequent quad is only reduced by a **factor of two** due to both the **reflected neutrons** and the high flux of **very energetic ones (>100 MeV).**

Heat deposition in the coils for 400 KW 48Ca beam





Coil Life Estimate

Using an average density of 10 g/cm^3 , 10 mW/cm^3 give a dose rate of **1 Gy/s**

With 10^7 s per year operation, this is **10 MGy** dose.

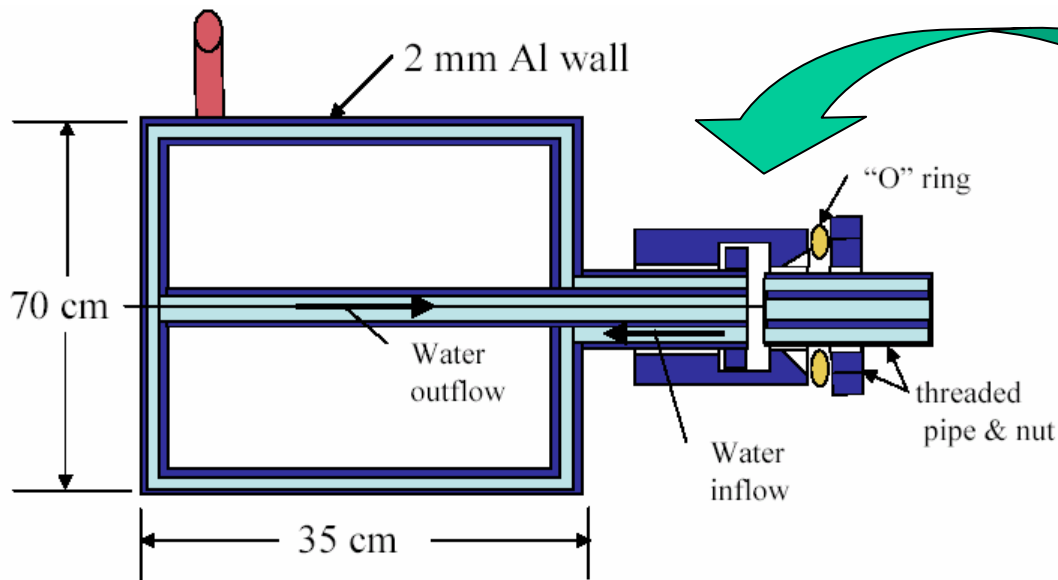
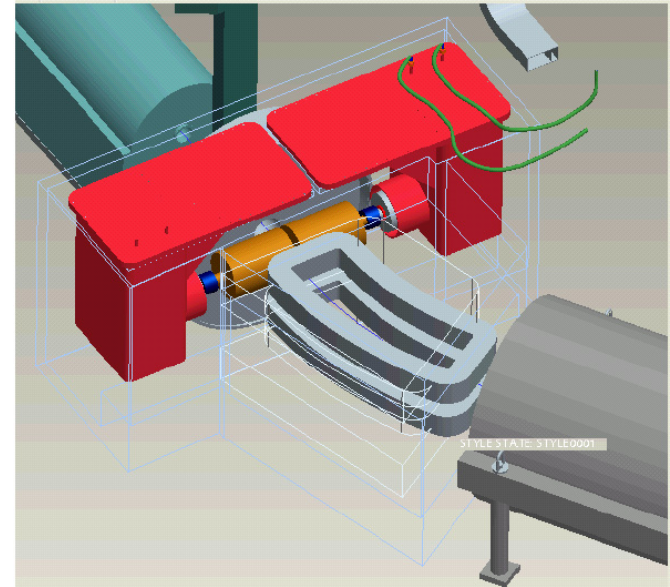
If HTS is as radiation resistant as Nb_3Sn (500 MGy), then coils last **50 years**.

Plan to test HTS 12/05 at LBNL with protons. It will be compared with Nb_3Sn , which has known tolerance.

Advanced beam dump designs



- To mitigate radiation damage, rotating beam dump concepts are being considered
- In particular, a rotating barrel-shaped dump has been designed capable of withstanding a 1cm-diameter beam spot
- U beam stops in cooling water, avoiding high DPA values in structural material

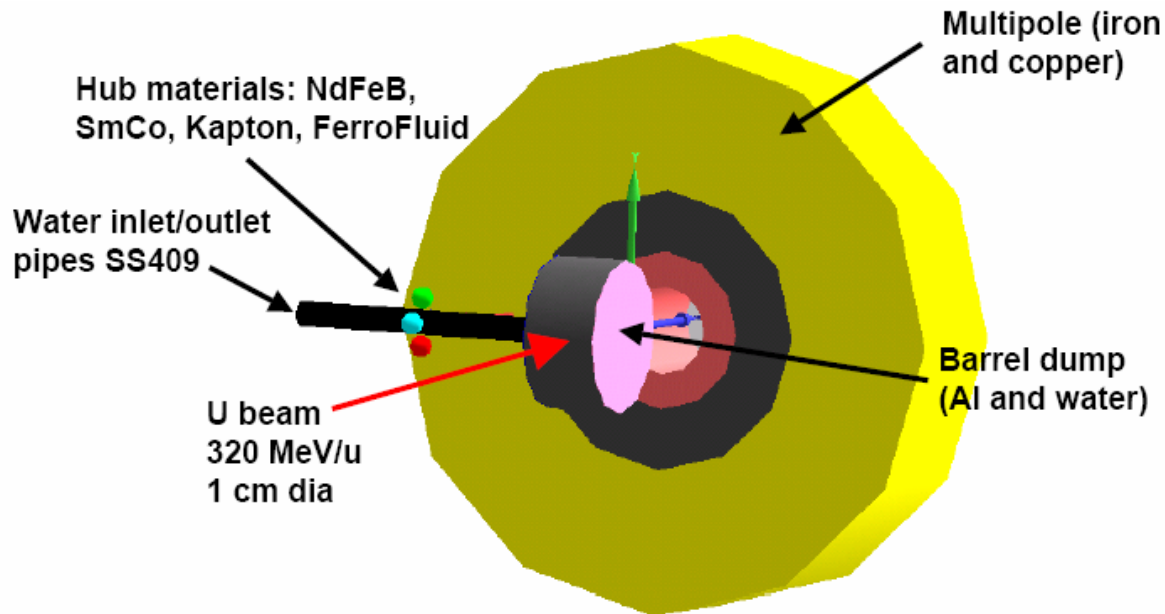


Need to address prompt and decay dose to sensitive components in rotating vacuum seal

Radiation transport results

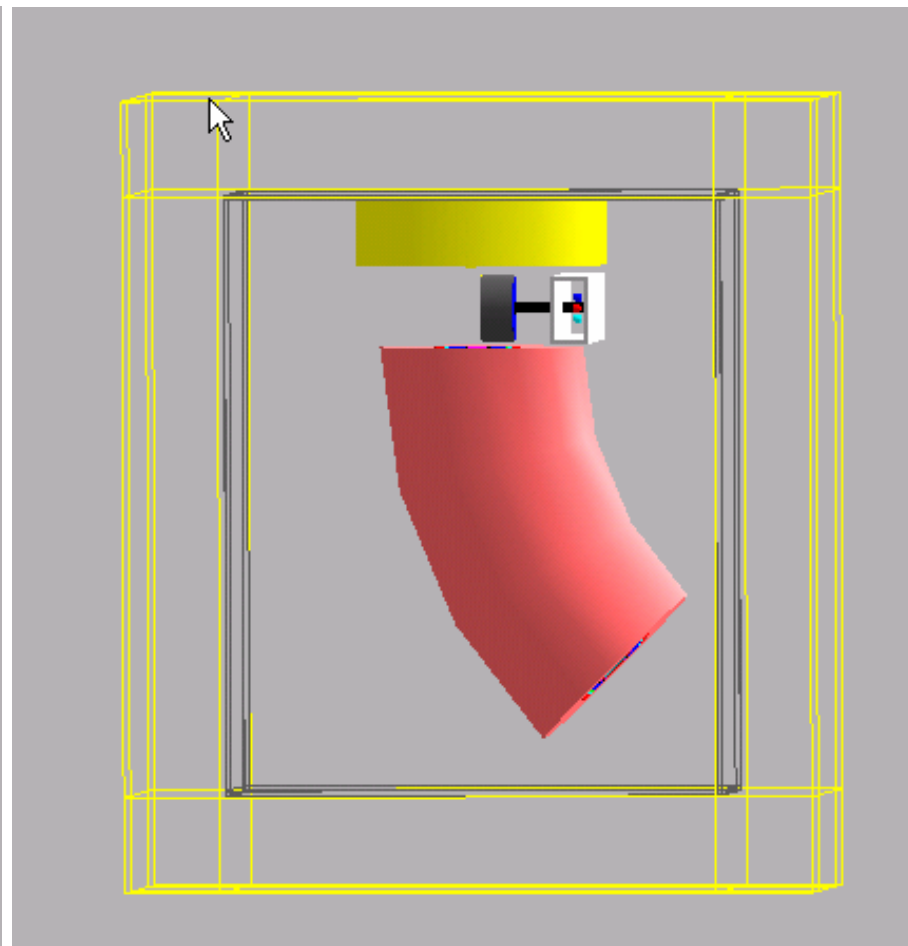
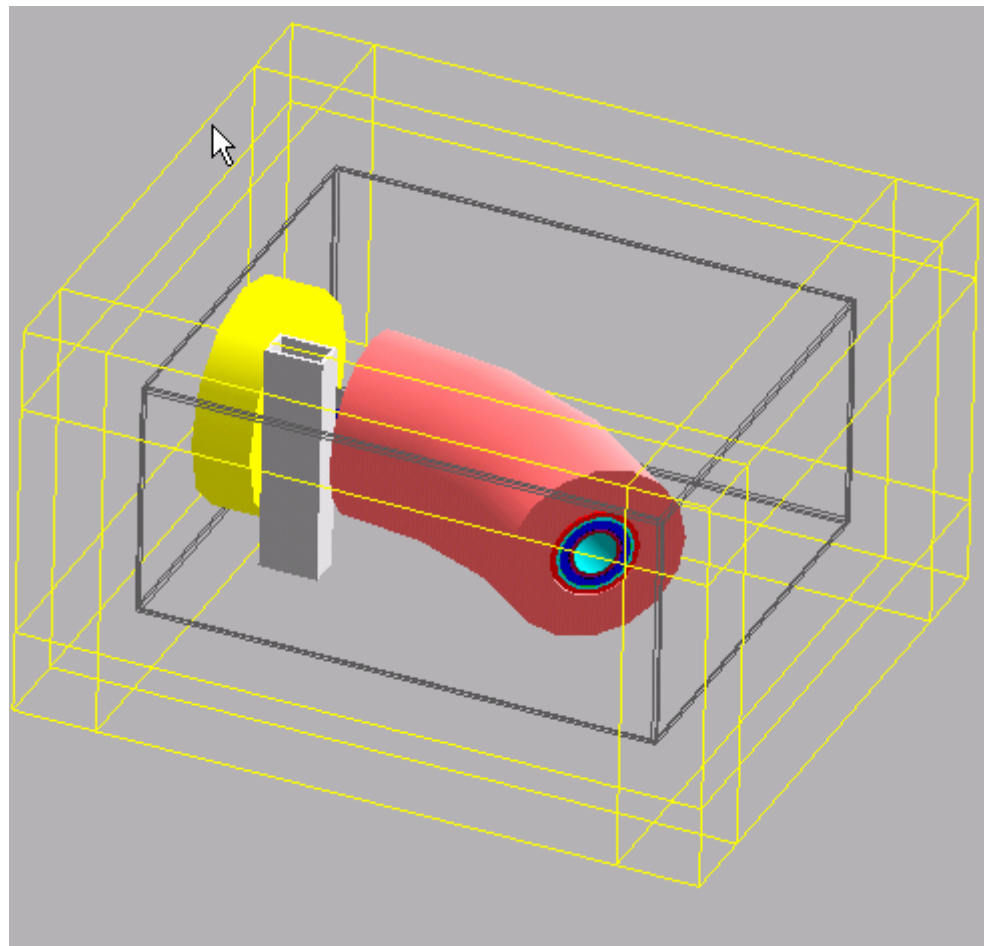


- We have used the heavy ion transport code PHITS to simulate particle transport in pre-separator area
- Model includes barrel beam dump, steel water inlet/outlet pipes, hub region with representative materials and downstream multipole magnet



- Assumed operation with a 320 MeV U beam with 1 cm-diameter spot size at a current of 3×10^{13} pps

Vacuum enclosure and dipole included



Radiation transport results: prompt dose and DPA

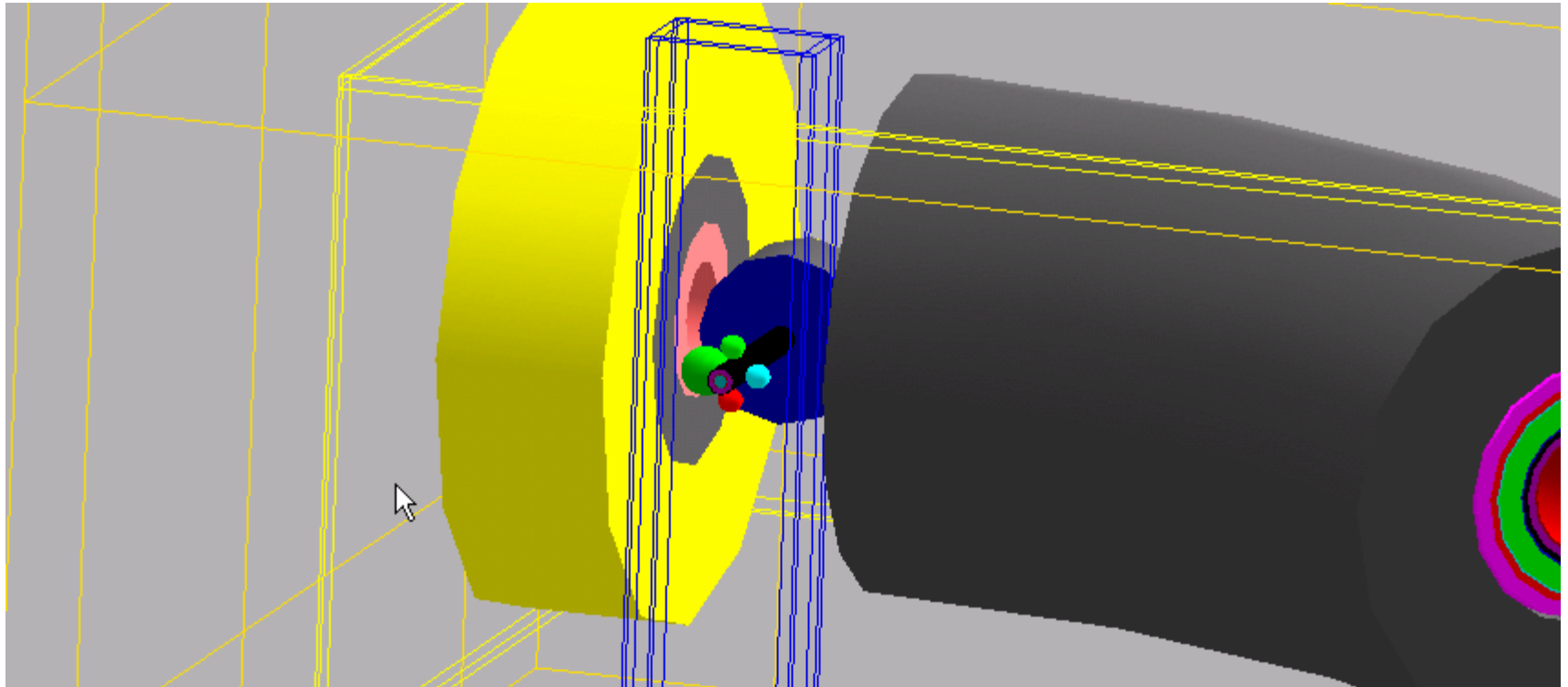


Material	Density (g/cc)	Effective dose (MGy/yr*)	Dose limit (MGy)	DPA/yr*
NdFeB	6	0.29	0.1	4.5E-06
SmCo	8.82	0.15	100	5.9E-06
Kapton	1.42	0.74	10	7.6E-07
FerroFluid	1.42	1.08	>1?	7.1E-07

*Assumed that fragmentation line is operating at full power for one-third of the calendar year

- **Dose to NdFeB magnet exceeds recommended limit after ~ 2 months of full power operation: shielding needed to extend lifetime**
- **DPA in the hub materials found to be negligible**
- **Maximum DPA rate in the Al barrel ~0.03 DPA/yr* (most of the primary beam stops in water); maximum DPA in multipole ~5x10⁻⁴ DPA/yr***
- **Peak energy deposition in multipole = 0.03 W/cc, with 2.1 kW total**

Borated (5 wt%) polyethylene shielding around NdFeB magnet (5 cm thick spherical shell)

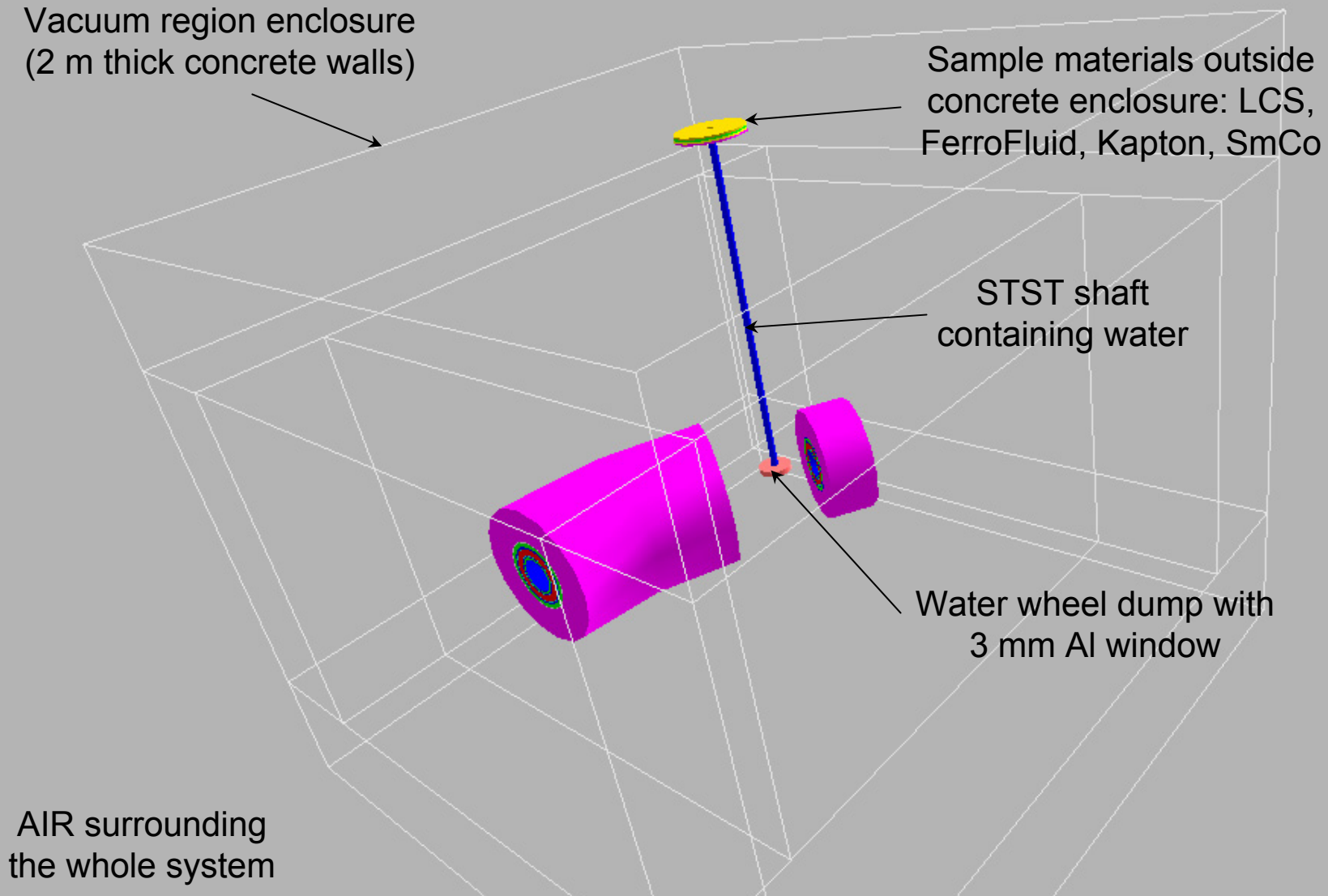


Material	Density (g/cc)	Effective dose (MGy/yr*)	Limit (MGy)
NdFeB	6	0.05	0.1

* Assumes operational for 1/3 of each year

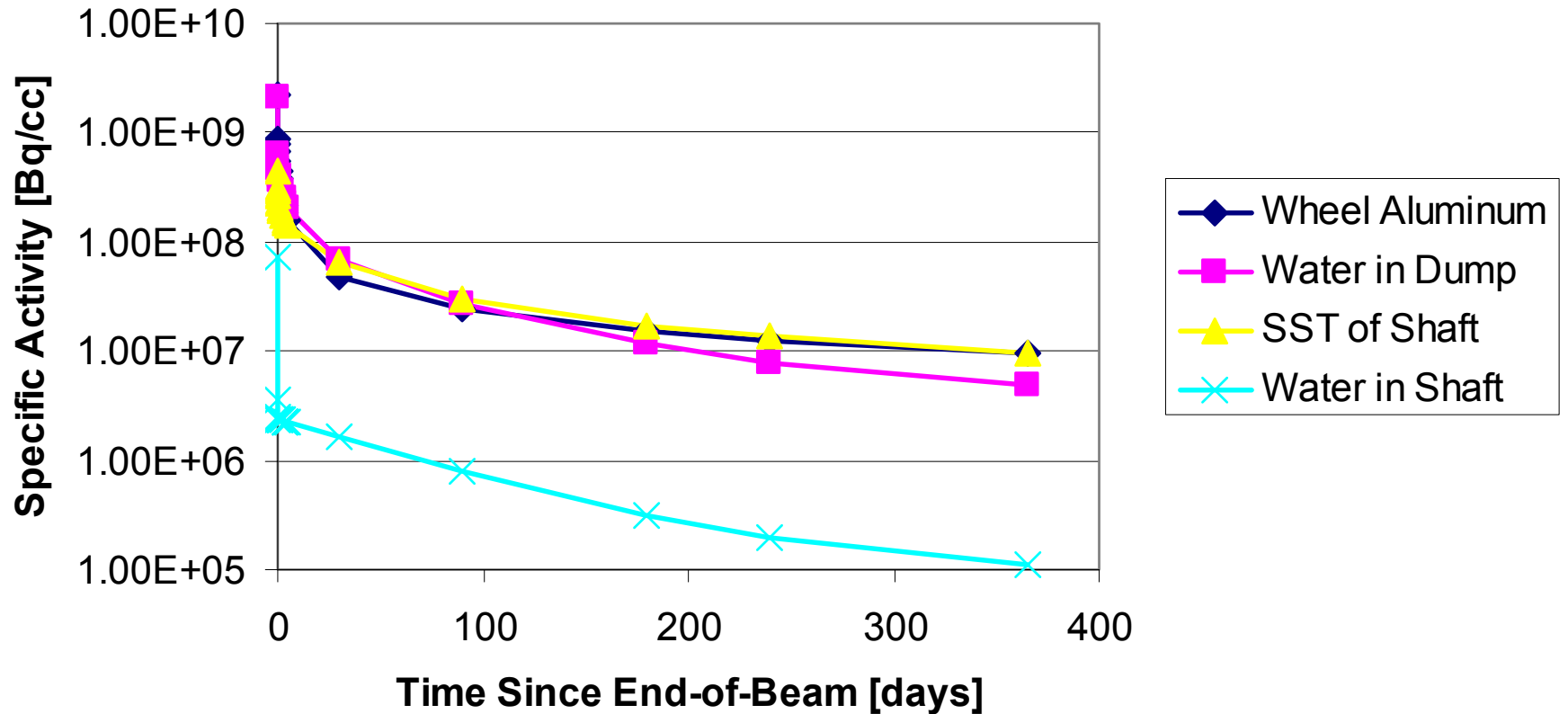
Simplified geometry model

^{136}Xe at 341 MeV/u, 3.74×10^{13} ions/sec



Investigate Radioactivity Inventory for Components

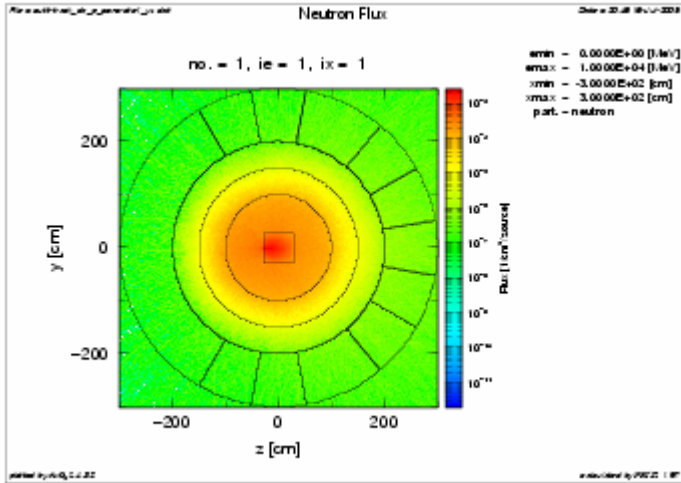
Wheel Beam Dump Activity



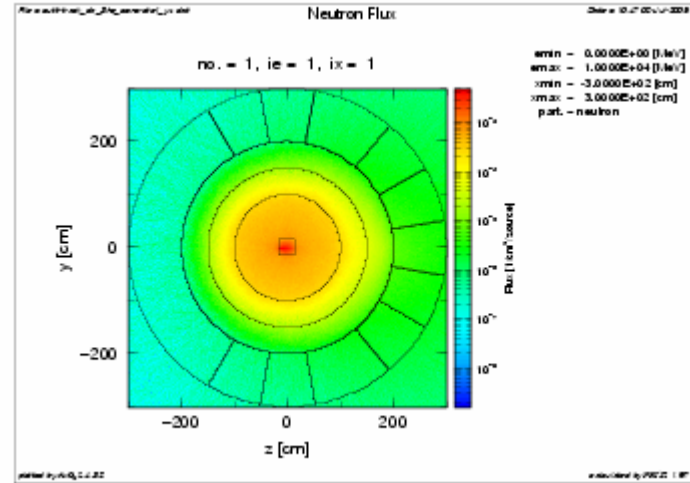
"Top-Ten"	90 days after EOB		Xe Beam					
no.	nuclide	[Bq/cc]	[Bq]	[%]	nuclide	[W/cc]	[W]	[%]
1	Be 7	1.25E+07	1.60E+11	47.2	Be 7	6.76E-07	8.62E-03	40.1
2	H 3	2.76E+06	3.52E+10	10.4	Y 88	1.82E-07	2.32E-03	10.8
3	I 125	2.16E+06	2.76E+10	8.2	Sb124	1.07E-07	1.37E-03	6.4
4	Xe127	1.05E+06	1.34E+10	4.0	Cs134	8.78E-08	1.12E-03	5.2
5	Y 88	4.21E+05	5.36E+09	1.6	Xe127	5.10E-08	6.50E-04	3.0
6	Rb 83	4.11E+05	5.25E+09	1.6	Cs136	4.04E-08	5.15E-04	2.4
7	In113m	3.53E+05	4.50E+09	1.3	Rb 83	3.47E-08	4.43E-04	2.1
8	Sn113	3.53E+05	4.50E+09	1.3	Sr 85	2.90E-08	3.69E-04	1.7
9	Sr 85	3.46E+05	4.42E+09	1.3	Rb 82	2.87E-08	3.66E-04	1.7
10	Kr 83m	3.38E+05	4.31E+09	1.3	Nb 95	2.40E-08	3.06E-04	1.4
	top-10 sum=	2.07E+07			top-10 sum=	1.26E-06		
	total activity	2.66E+07	[Bq/cc]	(3.38744E+11 [Bq])				
	total decay heat	1.69E-06	[W/cc]	(2.14929E-02 [W])				
	(beta)	7.56E-07	[W/cc]	(9.63692E-03 [W])	44.8%			
	(gamma)	9.30E-07	[W/cc]	(1.18560E-02 [W])	55.2%			
	(alpha)	2.02E-17	[W/cc]	(2.57106E-13 [W])	0.0%			
	activated atoms	2.51E+15	[cm**3]					
	(A=60-180:all)	2.79E+15	[cm**3]					
	(A=60-180:activated)	5.29E+14	[cm**3]					

"Top-Ten"	90 days after EOB		Uranium Beam					
no.	nuclide	[Bq/cc]	[Bq]	[%]	nuclide	[W/cc]	[W]	[%]
1	Be 7	7.49E+06	9.55E+10	38.1	Be 7	4.04E-07	5.15E-03	16.4
2	H 3	1.60E+06	2.04E+10	8.1	Po210	2.61E-07	3.33E-03	10.6
3	Nb 95	9.49E+05	1.21E+10	4.8	Nb 95	1.23E-07	1.57E-03	5.0
4	Zr 95	6.42E+05	8.19E+09	3.3	Zr 95	8.77E-08	1.12E-03	3.6
5	Rh103m	6.41E+05	8.17E+09	3.3	Rh106	7.55E-08	9.63E-04	3.1
6	Ru103	6.40E+05	8.17E+09	3.3	Po216	6.38E-08	8.13E-04	2.6
7	Y 91	5.55E+05	7.08E+09	2.8	Ru103	6.09E-08	7.77E-04	2.5
8	Sr 89	4.71E+05	6.01E+09	2.4	Rn220	5.91E-08	7.54E-04	2.4
9	Pa233	3.89E+05	4.97E+09	2.0	Po213	5.87E-08	7.49E-04	2.4
10	Po210	3.02E+05	3.85E+09	1.54	Y 91	5.39E-08	6.87E-04	2.19
	total activity	1.96E+07	[Bq/cc]	2.50435E+11 [Bq])				
	total decay heat	2.46E-06	[W/cc]	(3.14304E-02 [W])				
	(beta)	6.80E-07	[W/cc]	(8.66643E-03 [W])	27.57%			
	(gamma)	5.97E-07	[W/cc]	(7.61780E-03 [W])	24.24%			
	(alpha)	1.19E-06	[W/cc]	(1.51461E-02 [W])	48.19%			
	activated atoms	1.85E+15	[cm**3]					
	(A=60-180:all)	1.24E+15	[cm**3]					
	(A=60-180:activated)	3.16E+14	[cm**3]					

Proton, ^3He , ^{238}U Comparison



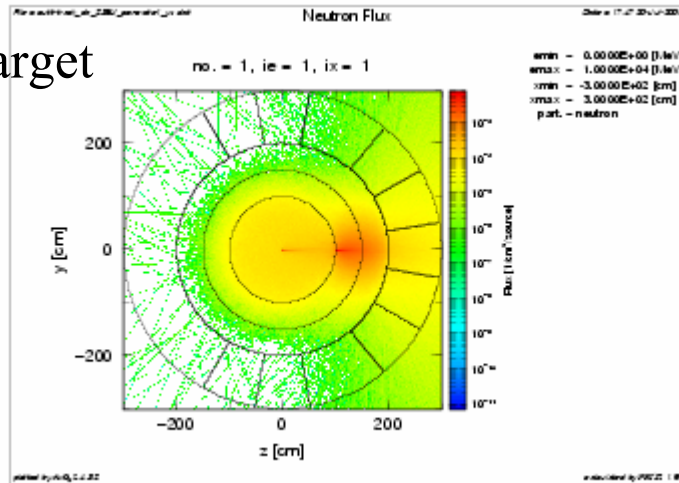
proton : 1 GeV



He-3 : 777 MeV/u

Beam on stopping Cu target

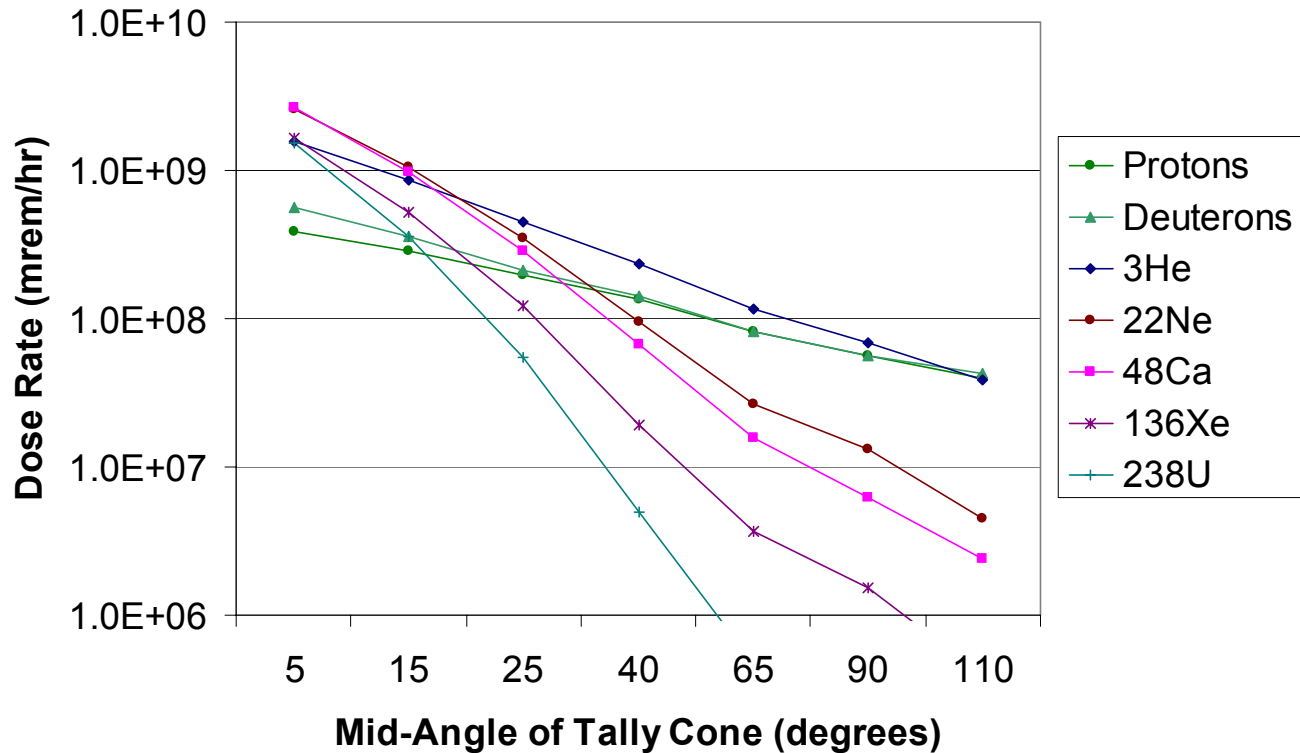
Beam direction
 →



U-238 : 400 MeV/u

Compare Effective Dose Equivalent for Different Beams

Compare Dose Rates
Outside of 1-meter Concrete Shield for:
Proton, d, 3He, 22Ne, 48Ca, 136Xe, 238U Beams
at 400 kW



Results for Proton Beam and Concrete Shield

Dose Rate Outside of Concrete Shielding 1 GeV protons on Stopping Cu Target; 400 kW

